



# **Cook Critical RP Survey Program**

A New, Systematic Approach To Radiation Protection Program Information Capture

Bob Hite - RP, Chemistry, Environmental Director American Electric Power - Cook Nuclear Plant

2013 ISOE International ALARA Symposium Tokyo, Japan, 27-29, 20<mark>13</mark>





## Cook Units 1 & 2 Background

- Westinghouse PWR Ice Condensers
- Unit 1 Commenced Commercial Operation in 1975
- Unit 2 Commenced Commercial Operation in 1978
- Total Construction Costs were low: \$1.3 Billion USD
- Plant Life Cycle Management Project Approved by Indiana and Michigan Rate Commissions for \$1.1 Billion USD for 20 additional years of operation (Unit 1 to 2034 & Unit 2 to 2037)



# RP, Chem & Environmental Director Background

- Arrived at Cook in fall 2011
- Previously RPM at Duane Arnold (BWR), Diablo Canyon (PWR), Prairie Island (PWR) & Bruce Power (CANDU).
- Expertise in Alpha Hazard Awareness,
   RP software data analysis,
   radiochemistry & cellular health effects

# **Cook Units Radiological Performance**

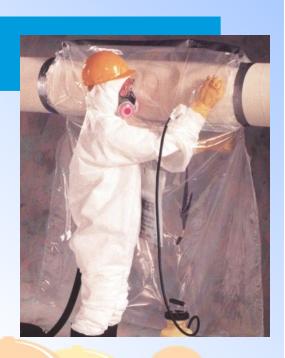
- Achieved lowest US & French PWR 3-year rolling average based on WANO data in 2012
- Cook was fourth quartile US PWR in 2002
- Unit 1 achieved 514 days of continuous operation prior to spring 2013 refueling outage
- Unit 2 has logged 455 days of continuous operation at of August 1, 2013





#### Concept

- The critical survey concept
  - Some important surveys during U2C20 were not obtained, OR
  - Were not of adequate quality
     AND
  - There was not an opportunity to re-do the survey
  - AND
  - Was important in program evaluation or decision making



## Concept



- The concept is to specifically:
  - Identify certain surveys as "critical"
  - Schedule the performance of the surveys with logic
  - Ensure surveys are completed the same shift
  - Enhance the level and timeliness of supervisory oversight over their conduct and timely review and approval



 Concept was introduced into radiation safety procedures ahead of U1C25 outage



#### **Critical Survey**



Defined by RP
Supervision as "a survey critical to plant radiological performance, industry or performance trending, or instrumental in the decision making process".



# **Examples of Critical Surveys**

- Initial steam generator bowl surveys
- EPRI standard radiation point surveys
- Containment entries at power surveys
- Before and after crud burst surveys
- Before and after refueling cavity decon
- Surveys for large dose estimates or risk
- Initial alpha characterisation surveys
- Any other survey selected by RP supervision

## "Critical Survey" Requirements

- Designated prior to performance to allow time for planning and discussion.
- Includes a pre job brief from RP supervisor to technician performing survey:
  - To include documentation (template), survey instruments and radiations to be measured.
- Survey to be completed by technician who has done survey before or briefed by technician or supervisor who has done it.
- Should be reviewed by supervision before work starts.
- Should be documented by technician by end of shift.





#### **Implementation in U1C25**

- 15 critical surveys defined for the outage
- Critical survey packages, including templates prepared
  - Tried to identify best past survey examples, or create new diagrams or maps when required
- Spreadsheet to identify and track status of critical survey completion created
  - Discussed at daily turnover meeting
  - Identified supervisor responsible for each critical survey on upcoming shift
- Master binder maintained of all critical survey results as they were performed



# **Critical surveys in U1C25**

- Initial Entry Surveys
- Post crud burst, post shielding surveys
- EPRI survey points
- Steam generator bowl survey
- Regen heat exchanger work
- Reactor cavity pre and post drain down (especially after EDM work)
- RHR IM-350 valve (alpha)
- RCDT waste drain valve (alpha)
- Pre transport surveys for LRSS bolt

## **Critical Surveys in U1C25**

- Used for alpha characterization surveys
  - Components that had not previously been characterized
  - Systems that had not been characterized
  - Items received from other facilities







- QRV-162
  - Three smears taken
    - Beta gamma levels 80 kdpm, 300 kdpm
      - Counted on AC3 probe, background
      - Ratio 5000:1
      - Could have used 3030/proteon for first smear
      - (could have been more accurate if counted lower beta gamma wipes)
  - Beta gamma level 25 kdpm
    - Counted on 3030, background
  - No air sample or lapel information on survey







- 1-WD-261 (at RCDT)
  - One smear from valve body
    - 29 dpm alpha, beta gamma to alpha ratio 990
  - One smear following mild abrasion
    - 71 dpm alpha, beta gamma to alpha ratio 544
  - Note that levels increased when surface was abraded indicating historical contamination
  - Air sample and lapel very low





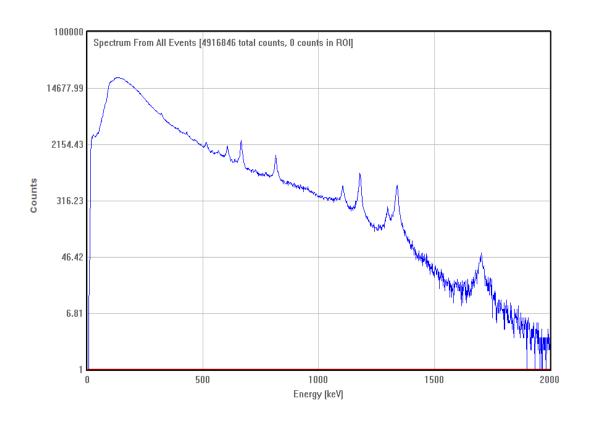
- RHR valve, 1-IMO-350
  - Five smears taken and monitored using AC3 probe - Contamination ranged from 100-220 cpm (526-800 dpm alpha)
  - Smears of smears taken from two of these
    - Activity 344 dpm and 237 dpm alpha on these
      - Equates to 3323 and 761 dpm on first smear
    - Beta gamma ratios from 361 to 1009
  - Air sample and lapel showed minimal activity



- CZT was used throughout the plant and identified a large amount of Cs-137 in beta gamma contamination
  - This confirms that alpha emitters very likely to be present in these areas/in the plant
- Further justifies the need for the enhanced alpha monitoring program



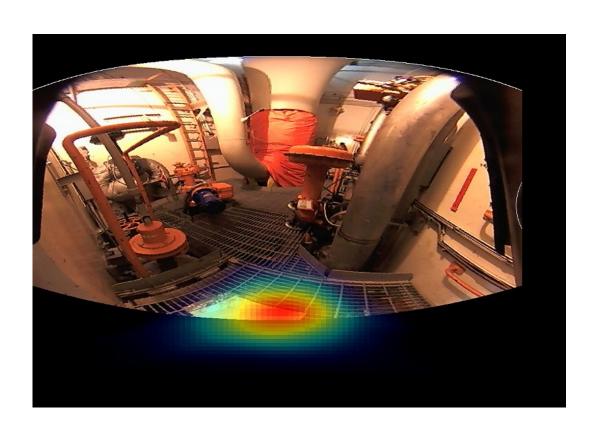
# **U1 E RHR Hx Mid-Outage**



- Measurement info: 04-02-2013 13:38:03 (22.2 min) U1 E RHR Hx
- Isotopes: Co-58, Fe-59, Co-60, Sb-124, Cs-137
- Trace amounts of: Cr-51, Nb-95, Zr-95



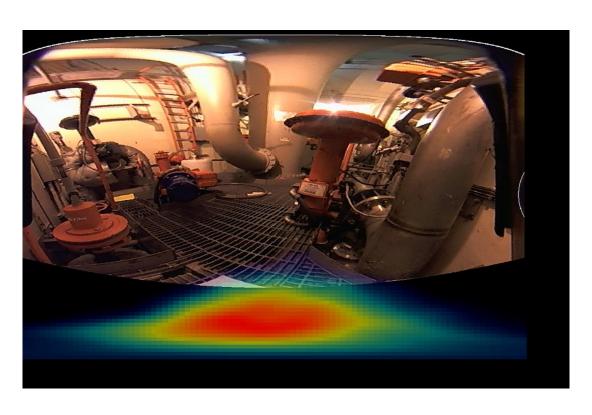
# **U1 E RHR Hx Mid-Outage**



- Measurement info: 04-02-2013 13:38:03 (22.2 min) U1 E RHR Hx
- Selected Isotopes in Post-Processed Imaging: Cs-137



# **U1 E RHR Hx Post-Outage**



- Measurement info: 05-16-2013 12:36:55 (16.5 min) U1 E RHR Hx GA
- Selected Isotopes in Post-Processed Imaging: Cs-137



# **U1 E RHR Time-lapse**

Cs-137

Pre-Outage Mid- Post-

Detector

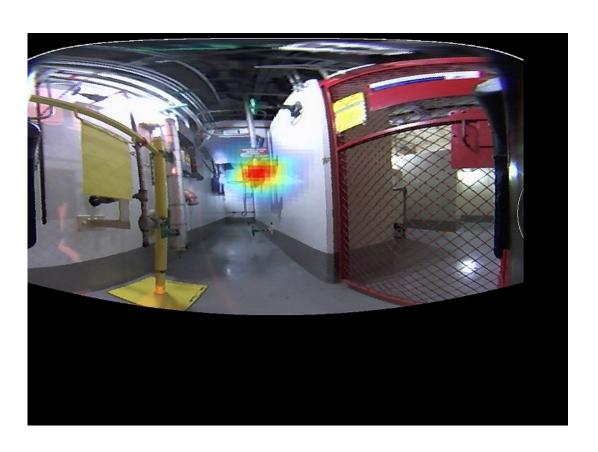


# 587' AUX BUILDING RADIATION AREA (RA)

Image of the hallway next to north waste evaporator filter/pump room. It is the only RA on the 587' level that is not located in a room. Current hot spot is 60 mR/hr contact and 5 mR/hr at 30 cm. Cobalt-60 is concentrated on valve 12-WD-322 in the waste disposal system. Cesium-137 is embedded in the floor, walls and piping.



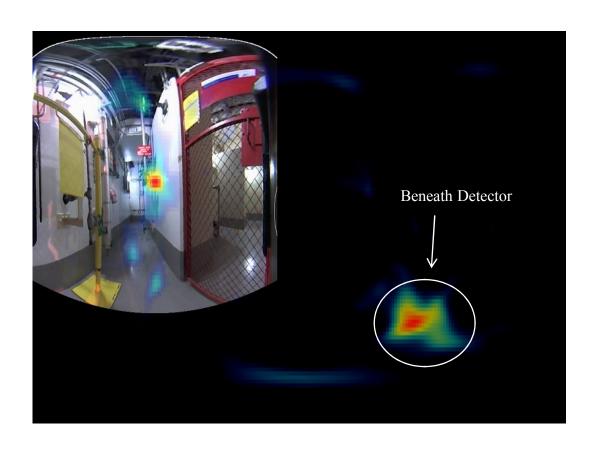
# **587' Aux RA**



- Measurement info: 06-03-2013 13:40:32 (11.6 min) N. Waste Evaporator Feed/Filter Pump Room
- Selected Isotopes in Post-Processed Imaging: Co-60



# 587' Aux RA



- Measurement info: 06-03-2013 13:40:32 (11.6 min) N. Waste Evaporator Feed/Filter Pump Room
- Selected Isotopes in Post-Processed Imaging: Cs-137

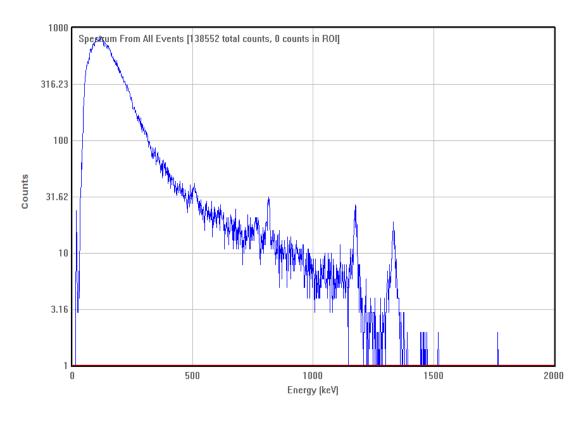


# 650' AUXILIARY BUILDING

120,000 ccpm personnel contamination occurred in a "clean areas" on the 650' aux building. This measurement located areas for follow-up smears by RP technicians. Two 2,000 ccpm particles and one 140,000 ccpm particle were found and removed.



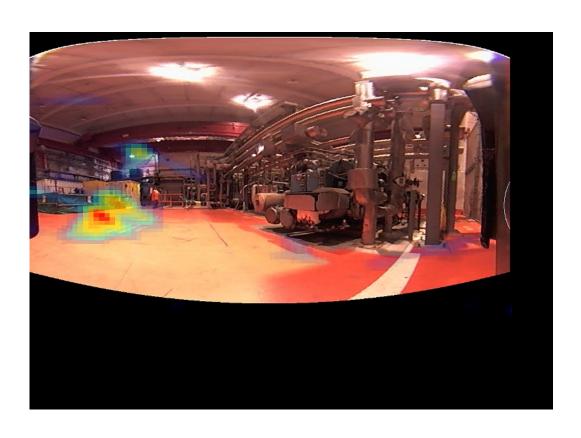
# 650' Aux Building



- Measurement info: 05-20-2013 15:24:09 (30.5 min) 650' Elevation O/S Vital Area
- Isotopes: Co-58, Co-60
- Trace Amounts of: Nb-95



# 650' Aux Building



- Measurement info: 05-20-2013 15:24:09 (30.5 min) 650' Elevation O/S Vital Area
- Selected Isotopes in Post-Processed Imaging: Co-58



# 650' Aux Building



- Measurement info: 05-20-2013 15:24:09 (30.5 min) 650' Elevation O/S Vital Area
- Selected Isotopes in Post-Processed Imaging: Co-60





#### **Critical Surveys Results**

- Increased overall quality of survey data
- Increased oversight over the performance of important surveys
  - They were <u>done</u> by <u>right person</u> at <u>right time</u> in the <u>right way</u>
- QA over dose rate surveys used to verify appropriateness of dose estimates
  - Used good survey data to lower dose estimate in one case
- Improved confidence in alpha monitoring data collected
- Demonstrated effectiveness of new process for cavity water clean up
- Feedback from supervisors and technicians very positive





## **Critical Survey Results**

- When choice made NOT to do a critical survey, then noticed drop in standards:
  - Reactor pit
    - Routine survey was relied upon
    - Two dose rate alarms found, and discovered routine survey was not comprehensive enough
    - Implementation of the critical survey concept in this area may have prevented this event
  - Steam generator nozzle dam box opening
    - Could not prepare before box was opened
    - Alpha surveys were conducted but not recorded



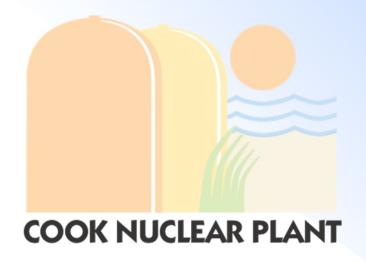


- Recording precisely how alpha monitoring was performed improves accuracy of the alpha data and provides additional information:
  - Smear of a smear in the RHR-IM-350 valve identified 344 dpm alpha, which was recorded on the survey as 344 dpm, but actually was 3323 dpm alpha if ratio back to first smear
  - Abrasive smear of RCDT waste valve showed alpha activity increased by 2.5 times and ratio decreased by a factor of two from surface smear
- Both issues were picked up from review of the "critical survey" performed for job coverage

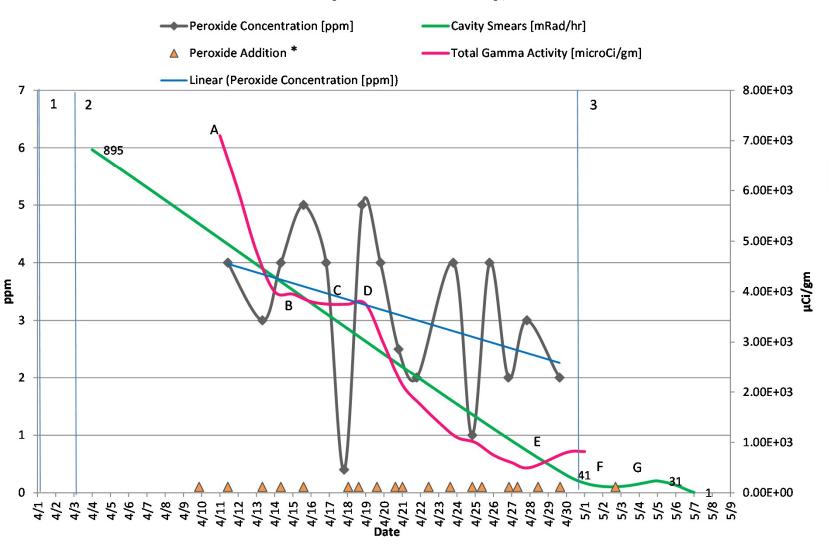
#### **Lessons Learned**

- Reactor cavity clean using peroxide addition and recirculation of cavity water through the spent fuel pool demineralizer using PRC-01 resin was effective
- See Graph





#### **Total Gamma Activity - Reactor Cavity/Refuel Canal 1**





#### **Conclusions**



- Critical survey concept is a useful tool to increase the oversight and therefore quality of survey data obtained
- Focus on survey quality for critical surveys will impact quality of all surveys as errors are identified and survey quality is improved
- Critical survey concept will continue to be used and refined further at Cook