## (4) Reprocessing Facility (radioactive gaseous waste)

Japan Atomic Energy Agency Reprocessing facility			Krypton [ <sup>85</sup> Kr] (Bq)	Iodine [ <sup>129</sup> I] (Bq)
	Reprocessing facility total		1.8E+10	6.6E+06
	Annual release control target values		8.9E+16	1.7E+09
Japan Nuclear Fuel Ltd. Reprocessing plant		Radioactive argon (Bq)	Krypton [ <sup>85</sup> Kr] (Bq)	Iodine [ <sup>129</sup> I] (Bq)
(Reprocessing facility)	Reprocessing facility total	-	N.D.	N.D.
	Annual release control target values	-	3.3E+17	1.1E+10

		Total radioactive particulate matter		
Japan Atomic Energy Agency Reprocessing facility		[total alpha] (Bq)		[total beta gamma] (Bq)
	Reprocessing facility total	N.D.		*1 4.7E+07
	Annual release control target values	*15 2.2E-08		*15 1.1E-04
		Other radionuclides (nuclides that emit alpha rays)	Breakdown of the left column (by nuclide)  Plutonium	Other radionuclides (nuclides that do not emit alpha
Japan Nuclear Fuel Ltd. Reprocessing plant		(Bq)	[Pu (α)] (Bq)	rays) (Bq)
(Reprocessing facility)	Reprocessing facility total	N.D.	N.D.	N.D.
	Annual release control target values	3.3E+08	-	9.4E+10

Notes: The radioactivity (Bq) of radioactive gaseous waste is obtained by multiplying the concentration of the radioactive material  $(Bq/cm^3)$  in the released gas by the amount of released gas.

Values lower than the detection limit of radioactivity are indicated as N.D.

The detection limits are as follows. (Bq/cm<sup>3</sup>)

Japan Atomic Energy Agency, Rep	processing Facility	Japan Nuclear Fuel Ltd., Reprocessing Plan	t (reprocessing facility)
<sup>14</sup> C :	4.0E-05 or less	Radioactive argon	: 1E-04 or less
Total radioactive particulate :	1.5E-10 or less	<sup>85</sup> Kr	: 2E-02 or less
matter (Total alpha rays)		$^{129}I$	: 4E-08 or less
		<sup>14</sup> C	: 4E-05 or less
		Other radionuclides (nuclides that emit alpha rays)	: 4E-10 or less
		(The value for all alpl	ha values was used.)
		Pu (a)	: 4E-10 or less
		Other radionuclides (nuclides that do not	en: 4E-9 or less
		(The value for all beta (gamn	na) values was used.)
		<sup>106</sup> Ru- <sup>106</sup> Rh	: 4E-9 or less
		(The values for particulate <sup>106</sup> Ru and volatile	<sup>106</sup> Ru are indicated.)
		$^{137}\text{Cs-}^{137\text{m}}\text{Ba}$	: 4E-9 or less
		$^{90}$ Sr- $^{90}$ Y	: 4E-10 or less

<sup>\*15:</sup> Mean concentration control target values (Bq/cm³) for three months

## (4) Reprocessing Facility (radioactive gaseous waste) (cont.)

Iodine [131] (Bq)	Tritium [³H] (Bq)	Carbon [ <sup>14</sup> C] (Bq)
*1 1.0E+10	6.0E+11	N.D.
1.6E+10	5.6E+14	5.1E+12
Iodine [ <sup>131</sup> I] (Bq)	Tritium [3H] (Bq)	Carbon [ <sup>14</sup> C] (Bq)
1.5E+05	2.4E+11	N.D.
1.7E+10	1.9E+15	5.2E+13

Breakdow	Breakdown of the left column (by nuclide)		
Strontium - Yttrium	Ruthenium - Rhodium	Cesium - Barium	
[ <sup>90</sup> Sr- <sup>90</sup> Y]	[ <sup>106</sup> Ru- <sup>106</sup> Rh]	$[^{137}Cs-^{137m}Ba]$	
(Bq)	(Bq)	(Bq)	
N.D.	N.D.	N.D.	
	-		